A free and open source computational tool for solving (nuclear-related) differential equations in the cloud

German (a.k.a. Jeremy) Theler

International Nuclear Atlantic Conference Round Table 4 ENFIR "Best Estimate codes and methods for Reactor Physics, Thermal Fluid Dynamics and Fuel Behavior"

3b3ae06-2021-12-01

FeenoX¹

a free no-fee no-X uniX-like finite-element(ish) computational engineering tool

Presentation slides https://www.seamplex.com/feenox -Sources-and full examples at https://github.com/gtheler/2021-brasil

¹FeenoX needs a logo.

CC-BY-SA 4.0

Why \rightarrow How \rightarrow What



https://www.youtube.com/watch?v=u4ZoJKF_VuA

CC-BY-SA 4.0



IAEA 2D/3D PWR benchmark (1976)



Two Group Cross Sections for Each Composition

Material	De	Σαρ	VΣte	Σ _{\$2.0-1}	
Evel 1	1,500	0.010	0.000	0.000	
Fuerr	0,400	0,085	0,135	0,020	
Eval 14Rod	1,500	0.010	0.000	0.000	
Fuertinou	0,400	0,130	0,135	0.020	
Euro/2	1,500	0.010	0,000	0.020	
Fueiz	0.400	0.080	0.135		
Deflecter	2,000	0,000	0,000	0.040	
Heliector	0.300	0.010	0.000	0,040	
DefeateriDad	2,000	0.000	0,000	0.040	
Reliectormou	0,300	0.055	0,000	0,040	

Figure 1: PARCS



Figure 2: milonga

Vertical PHWR (Atucha)





Steady-state core-level neutronic calculations







CC-BY-SA 4.0

Transient emergency boron injection—CFD



Transient emergency boron injection—core-level neutronics







Transient emergency boron injection—cell-level neutronics



A little bit of history—College (2004-2008) (v1)

- ${\scriptstyle \blacksquare} \ {\rm Solve} \ \dot{{\bf x}} = F({\bf x},t)$
 - a. program an *ad-hoc* numerical method
 - b. use a standard numerical library in C or Python, or
 - c. use a high-level system such as Octave, Maxima, etc.
- Point reactor equations solved "graphically" with non-free software

$$\begin{cases} \dot{\phi}(t) = \frac{\rho(t) - \beta}{\Lambda} \cdot \phi(t) + \sum_{i=1}^{N} \lambda_i \cdot c_i \\ \dot{c}_i(t) = \frac{\beta_i}{\Lambda} \cdot \phi(t) - \lambda_i \cdot c_i \end{cases}$$

I would rather write the equations in ASCII like

```
phi_dot = (rho-Beta)/Lambda * phi + sum(lambda[i], c[i], i, 1, N)
c_dot[i] = beta[i]/Lambda * phi - lambda[i]*c[i]
```

A little bit of history-Nuclear industry (2008-2014)



Modern/Advanced \neq Fortran 90

Spatial discretizations





wasora & milonga (v2)



Wasora's an advanced suite



- Coupled unstructured fine-mesh neutronics and thermal-hydraulics methodology using open software (2018)
- Open software one-step coupled neutronics and CFD thermalhydraulics calculation (2016)
- https://github.com/seamplex/milonga-2015-workshop
- Reactivity coefficient estimation by fuel temperature for Atucha II Nuclear Power Plant from neutron flux measurements (2014)
- On the design basis of a new core-level neutronic code written from scratch (2014)
- Neutron diffusion on unstructured grids: comparison between finite volumes and finite elements (2013)
- Geometric optimization of nuclear reactor cores (2013)
- Unstructured grids and the multigroup neutron diffusion equation (2013)

FeenoX (v3)

Software Requirement Specifications

https://seamplex.com/feenox/doc/srs.html

- Industrial-level: open source for V&V
- Extensible: free (as in freedom)
- Cloud-first: programatically-defined
- Arbitrarily scalable for large problems
- Flexible
- Web & mobile-based GUIs
- QA
 - Reproducibility & traceability
 - Automated testing
 - Bug reporting & tracking

V & V

Software Design Specifications

https://seamplex.com/feenox/doc/sds.html

- GPLv3+: it is about *freedom* not price
- No-GUI script-friendly GNU/Linux binary

**												
mesh data input	(*.msh) (*.dat) (*.fee)	} } }	input	>		FeenoX	>	output	{ { {	termi data post	nal files (vtk/msh)	
++												

- Scalability based on UNIX
 - PETSc/SLEPc (MPI)
 - Gmsh (Metis)
- Flexibility shown in "what"
- Web GUI: https://www.caeplex.com
- Regression testing make check
- https://github.com/seamplex/feenox
- V & V: TODO! (help appreciated)!













Point kinetics

$$\begin{cases} \dot{\phi}(t) = \frac{\rho(t) - \mathbf{B}}{\Lambda} \cdot \phi(t) + \sum_{i=1}^{N} \lambda_i \cdot c_i \\ \dot{c}_i(t) = \frac{\beta_i}{\Lambda} \cdot \phi(t) - \lambda_i \cdot c_i \\ \hline t \left[\mathbf{s} \right] - \rho(t) \left[\mathbf{pcm} \right] \end{cases}$$

<i>F</i> () <i>F</i> =	· [-]
0	0
0	5
10	10
10	30
0	35
0	100

for $0 < t < 100 \mbox{ starting from}$ steady-steate conditions at full power.

INCLUDE parameters.fee # kinetic parameters				
# our phase space is flux, precursors and reactivity PHASE_SPACE phi c rho				
<pre>end_time = 100 # we need a tighter error to handle small reactivities rel_error = 1e-7</pre>				
<pre># steady-state initial conditions rho_0 = 0</pre>				
FUNCTION react(t) DATA { 0 0 # in parm 5 0 10 10 30 10 35 0 100 0 }				
<pre># reactor point kinetics equations rho = le-5*react(t) # convert pcm to absolute phi_dot = (rho-Beta)/Lambda * phi + vecdot(lambda, c) c_dot[i] = beta[i]/Lambda * phi - lambda[i]*c[i] PRINT t phi rho</pre>				

\$ feenox reactivity-from-table.fee > flux.dat

Point reactor equations with $\rho(t)$ given as scattered data



Inverse kinetics



\$ feenox inverse-dae.fee flux.dat > inverse-dae.dat
\$ feenox inverse-integral.fee flux.dat > inverse-integral.dat



20/26

2D IAEA PWR Benchmark

<pre>PROBLEM neutron_diffusion 2D GROUPS 2 DEFAULT_ARGUMENT_VALUE 1 quarter # either quarter or eigth READ_MESH iaea-2dpwr-\$1.msh</pre>						
<pre># each material is to a physical entity in the geometry file Bg2 = 0.8e-4 # axial geometric buckling in the z direction MATERIAL fuell {</pre>						
$D_{1=1.5}$ $D_{2=0.4}$	Sigma $a^2=0.080+D^2(x,y)*Bg^2$	nuSigma f2=0.135 }				
MATERIAL fu D1=1.5 D2=0.4	el2 { Sigma_a1=0.010+D1(x,y)*Bg2 Sigma_a2=0.085+D2(x,y)*Bg2	Sigma_s1.2=0.02 nuSigma_f2=0.135 }				
MATERIAL fu	el2rod {					
D1=1.5 D2=0.4	Sigma_a1=0.010+D1(x,y)*Bg2 Sigma_a2=0.130+D2(x,y)*Bg2	Sigma_s1.2=0.02 nuSigma_f2=0.135				
<pre>MATERIAL reflector { D1=2.0 Sigma_a1=0.000+D1(x,y)*Bg2 Sigma_s1.2=0.04 D2=0.3 Sigma_a2=0.010+D2(x,y)*Bg2 }</pre>						
BC external vacuum=0.4692 # "external" is the name of the entity BC mirror # first is the name, second is the BC						
SOLVE_PROBL PRINT %.5f WRITE_MESH	EM # "keff = " keff # iaea-2dpwr-\$l.vtk phil phi2 #	solve! print keff write fluxes				

\$ gmsh -2 iaea-2dpwr-quarter.geo \$ [...] \$ gmsh -2 iaea-2dpwr-eighth.geo \$ [...] \$ feenox iaea-2dpwr.fee quarter keff = 1.02986 \$ feenox iaea-2dpwr.fee eighth keff = 1.02975



https://www.seamplex.com/feenox/examples/#iaea-2d-pwr-benchmark

2D IAEA PWR Benchmark

milonga's 2D LWR IAEA Benchmark Problem case #013

quarter-symmetry core meshed using delaunay (quads, $\ell_c = 2$) solved with finite volumes



160

40

(c) Flux distribution $\phi_{\alpha}(x, 0)$ along the x axis

milonga's 2D LWR IAEA Benchmark Problem case #018

quarter-symmetry core meshed using delaunay (quads, lc =2) solved with finite elements



CC-BY-SA 4.0

160

120

(d) Flux distribution $\phi_{\alpha}(x, x)$ along the diagonal

2D IAEA PWR Benchmark

milonga's 2D LWR IAEA Benchmark Problem case #053 eighth-symmetry core meshed using delaunay (quads, $\ell_c = 2$) solved with finite volumes



milonga's 2D LWR IAEA Benchmark Problem case #058

eighth-symmetry core meshed using delaunay (quads, $\ell_c = 2$) solved with finite elements



23/26



- One-group neutron transport
- The Stanford Bunny as the geometry
- S_2 method in 3D (8 angular directions)
- Finite elements for spatial discretization





Conclusions & TODO

- 21st century: cloud-first approach
- Stick to free and open source software
 - free \neq open source
- More formulations (milonga already had them)
 - FEM transport
 - FVM diffusion & transport
- Massive parallelization
 - MPI through PETSc/SLEPc
 - Metis through Gmsh
- Documentation
 - https://www.seamplex.com/feenox/doc
 - https://www.seamplex.com/feenox/examples
 - Come up with a nice logo
- Build a community!
 - https://github.com/seamplex/feenox/discussions



https://www.youtube.com/watch?v =Q-IKK4A2OzA